June 30, 1985 NRC/TMI-86-063

MEHORANDUM FOR:

Harold R. Denton, Director

Office of Nuclear Reactor Regulation

Frank J. Miraglia, Director Division of PWR Licensing-B

FROM:

William D. Travers, Director

TMI-2 Cleanup Project Directorate

SUBJECT:

HRC TMI-2 CLEANUP PROJECT DIRECTORATE WEEKLY STATUS

REPORT FOR JUNE 23 - JUNE 29, 1986

#### 1. DEFUELING

The first rail-mounted shipping cask was loaded with seven defueling canisters leaving 36 canisters stored in the fuel pool. The total weight of the debris contained in the seven canisters is 2400 lbs. Approximately 51,670 lbs. of debris have been loaded into canisters

and transferred to the fuel storage pool to date.

The shielded work platform on top of the reactor vessel is currently dedicated to core stratification sample acquisition (core bore) activities. The core boring rig was being aligned as of Monday, June 30, 1986. The core bore is designed to obtain full length samples of the reactor core from the debris bed to the elliptical flow distributor plate. Information gained from drilling regarding the hardness, ductility, and friability of the core material will be used to aid future defueling activities. The information will be useful to defueling tool development and defueling strategy. The core bore samples (approximately 21" in diameter and 8 ft. long) will be shipped to the Idaho National Engineering Laboratory for examination. Core bore activities are currently planned to continue for approximately four weeks.

On June 21 - 26, 1986, NRC's Vendor Inspection Branch performed a special inspection at Nuclear Pacific Corporation, Federal May, Washington, of activities and records pertaining to non-destructive examination and testing of the two NUPAC 125-B fuel debris shipping containers. A representative of the TMI-2 Cleanup Project Directorate participated in this effort. Areas reviewed included leak testing, radiography, ultrasonic testing, penetrant testing, and pressure testing of the containers. Vendor Inspection Branch will publish a report of the details of the inspection in

approximately three weeks.

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## 2. PLANT STATUS

- The reactor remains in long term cold shutdown, vented to the atmosphere. Core cooling is by natural heat loss to ambient building atmosphere. The average incore thermocouple reading is 78°F.
- The airborne radioactivity on the defueling platform is about 3 E-7 uCi/cc Tritium and 1.1 E-11 uCi/cc particulates, predominately Cesium-137 and Strontium-90. The platform is mounted above the modified internals indexing fixture which is mounted over the reactor vessel. These provide about 15 feet of water over the core region and 6 feet over the carousel holding the defueling canisters.

### 3. WASTE MANAGEMENT

NRC FORM 318 (10 80) NRCM 0240

 The Submerged Demineralizer System (SDS) completed processing batch S-133. SDS processing to date has been 4.098.333 gallons.

 EPICOR II completed processing batch 293, 294, 295, and 296. The system has processed 3,002,840 gallons to date.

### 4. ENVIRONMENTAL MONITORING

 US Environmental Protection Agency (EPA) sample analysis results show that TMI site liquid effluents are in accordance with regulatory limits, NRC requirements, and The City of Lancaster Agreement.

 The Lancaster water sample taken at the water works river intake and analyzed by EPA consisted of a seven day composite sample taken June 8 - 14, 1986. A gamma scan detected no reactor related activity.

 TMI water samples taken by EPA at the plant discharge (includes Units 1 and 2) to the river consisted of seven daily composite samples taken from June 7 - 14, 1986. A gamma scan detected no reactor related activity.

 The EPA analysis of the NRC outdoor air sample for the period June 19 - 26, 1986 showed that concentrations of Cs-137 and I-131 were below the lower limit of quantitative detectability for the system.

# 5. AUXILIARY AND FUEL HANDLING BUILDING ACTIVITIES

Assembly of the desludging system continued.

 Hands-on decontamination was conducted in the Westinghouse Valve Room at the 281 ft. elevation of the Fuel Handling Building.

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#### 6. NRC EVALUATIONS IN PROGRESS

- Technical Specification Change Request number 49, and 51.
- Recovery Operations Plan Change number 31 and 36.
- Solid Waste Facility Technical Evaluation Report.
- Reactor Building Sump Criticality Safety Evaluation Report. Defueling Canister Technical Evaluation Report, Revision 2.
- Extended Core Stratification Sample Acquisition Activity.
- Heavy Load Safety Evaluation Report, Revision 3.
- Defueling Safety Evaluation Report, Revision 10.

original signed by Curt Cowgill for:

Hilliam D. Travers Director THI-2 Cleanup Project Directorate

State Liaison, RI TMI HQ r/f

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